
**GOVERNMENT NOTICES
GOEWERMENTSKENNISGEWINGS**

**DEPARTMENT OF LABOUR
DEPARTEMENT VAN ARBEID**

No. R. 396

28 April 2006

**NOTICE OF EXEMPTION IN TERMS OF SECTION 40(1) OF THE
OCCUPATIONAL HEALTH AND SAFETY ACT, 1993**

DRIVEN MACHINERY REGULATIONS 18(5)

Under section 40 (3) (b) of the Occupational Health and Safety Act, 1993 (Act No. 85 of 1993), I Jacob Pannye Malatse appointed as chief inspector in terms of section 27(1) of the said Act, and by virtue of the power delegated to me by the Minister of Labour in terms of section 42 (1) of the Act, hereby grant exemption in terms of section 40 (1) from amended regulation 18(5) of the Driven Machinery Regulations published in government notice R.158 on 18 February 2005, until 29 September 2006 to all approved lifting machinery entities to perform load test on lifting machines without a lifting machine inspector who is registered by Engineering Council of South Africa

J. P. Malatse

Chief Inspector

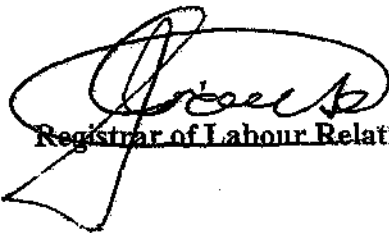
No. R. 402

28 April 2006

LABOUR RELATIONS ACT, 1995

**CANCELLATION OF REGISTRATION OF AN EMPLOYERS'
ORGANISATION
SOUTH AFRICAN LUMBER MILLERS' ASSOCIATION**

I, Johannes Theodorus Crouse, Registrar of Labour Relations, hereby notify, in terms of section 109(2) of the Labour Relations Act, 1995, that I have cancelled the registration of the **South African Lumber Millers' Association (LR 2/6/3/155)** with effect from 10 April 2006.



Registrar of Labour Relations

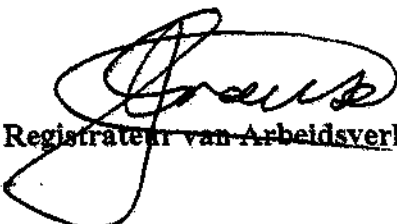
No. R. 402

28 April 2006

WET OP ARBEIDSVERHOUDINGE, 1995

**INTREKKING VAN REGISTRASIE VAN 'N WERKGEWERSORGANISASIE
SOUTH AFRICAN LUMBER MILLERS' ASSOCIATION**

Ek, Johannes Theodorus Crouse, Registrateur van Arbeidsverhoudinge, maak hierby, ingevolge artikel 109(2) van die Wet op Arbeidsverhoudinge, 1995, bekend dat die registrasie van die **South African Lumber Millers' Association (LR2/6/3/155)** met ingang van 10 April 2006 ingetrek is.



Registrateur van Arbeidsverhoudinge

**DEPARTMENT OF MINERALS AND ENERGY
DEPARTEMENT VAN MINERALE EN ENERGIE**

No. R. 388

28 April 2006

NATIONAL NUCLEAR REGULATOR ACT, 1999 (ACT NO. 47 OF 1999)

**REGULATIONS IN TERMS OF SECTION 36, READ WITH SECTION 47 OF
THE NATIONAL NUCLEAR REGULATOR ACT, 1999 (ACT NO. 47 OF 1999),
ON SAFETY STANDARDS AND REGULATORY PRACTICES**

Under section 36, read with section 47 of the National Nuclear Regulator Act, 1999 (Act No. 47 of 1999), I B. L. Hendricks, Minister of Minerals and Energy, on the recommendation of the Board of Directors of the National Nuclear Regulator, hereby make the regulations in the Schedule.

**B L HENDRICKS
MINISTER OF MINERALS AND ENERGY**

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Section 1**Definitions****1. Terms defined in the Act**

In these regulations, any word or expression to which a meaning has been assigned in the *Act*, shall have the meaning so assigned.

2. Terms not defined in the Act

In these regulations, unless the context indicates otherwise:

- (i) "*absorbed dose*" means the fundamental dosimetric quantity D expressed in the unit $J \cdot kg^{-1}$, termed the gray (Gy), defined as:

$$i. D = \frac{d\bar{\epsilon}}{dm}$$

where $d\bar{\epsilon}$ is the mean energy imparted by ionizing *radiation* to matter in a volume element and dm is the mass of matter in the volume element;

- (ii) "*assessment*" means the process, and the result, of analysing systematically the hazards associated with *sources* and *actions*, and associated protection and safety measures, aimed at quantifying performance measures for comparison with criteria;
- (iii) "*authorised*" means permitted in writing by the *Regulator*;
- (iv) "*authorised action*" means an *action authorised* in terms of the National Nuclear Regulator Act, 1999 (Act No. 47 of 1999);
- (v) "*average member of the critical group*" means the individual receiving the average *effective dose* or *equivalent dose* (as applicable) in the *critical group*;
- (vi) "*becquerel*" (Bq) means the unit of *radioactivity* in nuclear transformations (or disintegrations) per second;
- (vii) "*clearance*" means removal of *radioactive materials* or radioactive objects within *actions authorised* by a *nuclear installation licence*, *nuclear vessel licence* or *certificate of registration* from any further control by the *Regulator*;
- (viii) "*collective dose*" means an expression for the total *radiation dose* incurred by a population, defined as the product of the number of individuals exposed to a *source* and their average *radiation dose*. The *collective dose* is expressed in person-sievert (person.Sv) (see *collective effective dose*);
- (ix) "*collective effective dose*" means the total *effective dose* incurred by a population, being the sum of all the individual *effective doses* to members of the population. Mathematically, the total *effective dose* to a population, S , is calculated as:

$$i. S = \sum_i E_i \cdot N_i$$

where E_i is the average *effective dose* in the population subgroup i and N_i is the number of individuals in the subgroup. It can also be defined by the integral:

$$\text{ii. } S = \int_0^{\infty} E \frac{dN}{dE} dE$$

where $\frac{dN}{dE} dE$ is the number of individuals receiving an *effective dose* between E and $E+dE$.

The *collective effective dose* S_k committed by an event, a decision or a finite portion of an action k , during a time interval T , is given by:

$S_k = \int_0^T \dot{S}_k(t) dt$ where $\dot{S}_k(t)$ is the *collective effective dose rate* at time t caused by k ;

- (x) "*committed equivalent dose*" means the quantity $H_T(\tau)$, defined as:

$$\text{i. } H_T(\tau) = \int_{t_0}^{t_0+\tau} \dot{H}_T(t) dt$$

where t_0 is the time of *intake*, $\dot{H}_T(t)$ is the *equivalent dose rate* at time t in organ or tissue T and τ is the time elapsed after an *intake* of radioactive substances. When τ is not specified, it will be taken to be 50 years for adults and to age 70 years for *intakes* by children;

- (xi) "*critical group*" means a group of members of the public which is reasonably homogeneous with respect to its *exposure* for a given *radiation source* and given *exposure pathway* and is typical of individuals receiving the highest *effective dose* or *equivalent dose* (as applicable) by the given *exposure pathway* from the given *source*;
- (xii) "*decommissioning*" means administrative and technical actions taken to allow the removal of all of the regulatory controls from a facility (except for a repository which is closed and not decommissioned);
- (xiii) "*defence in depth*" means the application of more than a single protective measure for a given *radiation* or *nuclear safety* objective, so that the objective is achieved even if one of the protective measures fails;
- (xiv) "*discharge*" means a planned and controlled release of *radioactive nuclides* to the environment;
- (xv) "*disposal*" means the emplacement of *radioactive waste* in an approved, specified facility without the intention of retrieval and "*dispose of*" has the corresponding meaning;
- (xvi) "*dose*" means the amount of *radiation* received, where the use of a more specific term such as "*effective dose*" or "*equivalent dose*" is not necessary for defining the quantity of interest;
- (xvii) "*dose constraint*" means a prospective and *source-related* restriction on the individual *dose* arising from the predicted operation of the *authorised action* which serves exclusively as a bound on the optimisation of *radiation protection* and *nuclear safety*:

- (a) to limit the range of options considered in the optimisation process, and
 (b) to restrict the *doses* via all *exposure pathways* to the *average member of the critical group*, in order to ensure that the sum of the *doses* received by that individual from all controlled *sources* remains within the *dose limit*, and which, if found retrospectively to have been exceeded, should not be regarded as an infringement of regulatory requirements but rather as a call for the *reassessment* of the optimisation of *radiation protection*.
- (xviii) "*dose limit*" means the value of *effective dose* or *equivalent dose* to individuals from *actions authorised by a nuclear installation licence, nuclear vessel licence or certificate of registration*, that must not be exceeded;
- (xix) "*effective dose*" means the quantity E expressed in the unit $\text{J}\cdot\text{kg}^{-1}$, termed the sievert (Sv), defined as the summation of the tissue *equivalent doses*, each multiplied by the appropriate *tissue weighting factor*:

$$\text{i. } E = \sum_T w_T \cdot H_T$$

where H_T is the *equivalent dose* in tissue T and w_T is the *tissue weighting factor* for tissue T ; from the definition of *equivalent dose*, it follows that:

$$\text{ii. } E = \sum_T w_T \cdot \sum_R w_R \cdot D_{T,R}$$

where w_R is the *radiation weighting factor* for *radiation* R and $D_{T,R}$ is the average *absorbed dose* in the organ or tissue T ;

- (xx) "*emergency planning*" means the process of developing and maintaining the capability to take actions that will mitigate the impact of an emergency on persons, property or the environment;
- (xxi) "*emergency preparedness*" means the capability to promptly take actions that will effectively mitigate the impact of an emergency on persons, property or the environment;
- (xxii) "*emergency response*" means the performance of actions to mitigate the impact of an emergency on persons, property or the environment;
- (xxiii) "*equivalent dose*" means the quantity $H_{T,R}$ expressed in the unit $\text{J}\cdot\text{kg}^{-1}$, termed the sievert (Sv), defined as:

$$\text{i. } H_{T,R} = D_{T,R} \cdot w_R$$

where $D_{T,R}$ is the *absorbed dose* delivered by *radiation* type R averaged over a tissue or organ T and w_R is the *radiation weighting factor* for *radiation* type R ; when the *radiation* field is composed of different *radiation* types with different values of w_R , the *equivalent dose* is:

$$\text{ii. } H_T = \sum_R w_R \cdot D_{T,R}$$

- (xxiv) "*environmental monitoring*" means the measurement of external *dose* rates due to *sources* in the environment and of *radioactive nuclide* concentrations in environmental media;
- (xxv) "*exclusion*" means *exclusion from the scope of regulatory control*;

- (xxvi) "*exemption*" means the determination by the *regulator* that an *action* need not be subject to some or all aspects of regulatory control on the basis that the exposure (or potential exposure) due to the *action* is too small to warrant the application of those aspects;
- (xxvii) "*exposure*" means the act or condition of being subject to irradiation;
- (xxviii) "*exposure pathway*" means a route by which *radioactive material* can reach or irradiate humans;
- (xxix) "*fertile*" means nuclear material which can be converted into material which is capable of nuclear fission;
- (xxx) "*fissile*" means material that undergo fission by neutrons of all energies;
- (xxxi) "*IAEA*" means the International Atomic Energy Agency;
- (xxxii) "*individual monitoring*" means *monitoring* using measurements by equipment worn by individual workers, or measurements of quantities of *radioactive materials* in or on their bodies;
- (xxxiii) "*institutional control*" means control of a waste *site* (for example, *disposal site*) by a statutory authority or institution; this control may be active (*monitoring, surveillance, remedial work*) or passive (land use control) and may be a factor in the design of a nuclear facility (for example, near surface *disposal facility*);
- (xxxiv) "*intake*" means the process of taking *radioactive nuclides* into the body by inhalation or ingestion or through the skin;
- (xxxv) "*monitoring*" means the continuous or periodic measurement of radiological and other parameters or determination of the status of a system;
- (xxxvi) "*normal operational exposure*" means an *exposure* which is expected to be received under normal operating conditions, including possible minor mishaps that can be kept under control;
- (xxxvii) "*nuclear safety*" means the achievement of safe operating conditions, prevention of *nuclear accidents* or mitigation of *nuclear accident* consequences, resulting in the protection of workers, the public and the environment against the potential harmful effects of ionizing radiation or *radioactive material*;
- (xxxviii) "*occupational exposure*" means *exposure* of a worker in the course of his or her work in excess of an annual *effective dose* of 1 mSv in addition to natural background *radiation*, and "occupationally exposed" has the corresponding meaning;
- (xxxix) "*operational safety assessment*" means a *safety assessment* undertaken during operations;
- (xl) "*prior safety assessment*" means a *safety assessment* undertaken prior to commencement of operations;
- (xli) "*radiation*" means *ionising radiation*;
- (xlii) "*radiation protection*" means the protection of people from the effects of *exposure to ionising radiation*, and the means for achieving this;
- (xliii) "*radiation weighting factor*" means a multiplier of *absorbed dose* used for *radiation protection* purposes to account for the relative effectiveness of different types of *radiation* in inducing health effects, the value of which is that specified in

the *IAEA International Basic Safety Standards for Protection against Ionizing Radiation and for the Safety of Radiation Sources*;

- (xlv) "**radioactive waste**" means any material, whatever its physical form, remaining from an *action* requiring a *nuclear installation licence*, *nuclear vessel licence* or *certificate of registration* and for which no further use is foreseen, and that contains or is contaminated with *radioactive material* and does not comply with the requirements for *clearance*;
- (xlv) "**radioactive waste acceptance criteria**" means the quantitative or qualitative criteria, specified by the operator and approved by the *regulator*, for *radioactive waste* to be accepted by the operator of a repository for disposal, or by the operator of a storage facility for storage;
- (xlvi) "**radon**" means the isotope ^{222}Rn of the element of atomic number 86;
- (xlvii) "**registration**" means the granting of a *certificate of registration*;
- (xlviii) "**risk**" means (qualitatively expressed) the probability of a specified health effect occurring in a person or group as a result of *exposure to radiation* or (quantitatively expressed) a multiattribute quantity expressing hazard, danger or chance of harmful or injurious consequences associated with actual or potential *exposures* relating to quantities such as the probability that specific deleterious consequences may arise and the magnitude and character of such consequences;
- (xlix) "**risk assessment**" means an *assessment* of the radiological risks associated with normal operation and potential accidents involving a source or *action*;
- (l) "**safety assessment**" means an analysis to evaluate the performance of an overall system and its impact, where the performance measure is radiological impact or some other global measure of impact on safety;
- (li) "**safety culture**" means the assembly of characteristics and attitudes in organizations and individuals which establishes that, as an overriding priority, protection and safety issues receive the attention warranted by their significance;
- (lii) "**source**" means anything that may cause *radiation exposure*, such as by emitting *ionising radiation* or releasing radioactive substances or materials; a complex or multiple installation situated at one location or *site* may, as appropriate, be considered as a single *source* for the purposes of application of these regulations;
- (liii) "**storage**" means the holding of spent (used) nuclear fuel or *radioactive waste* in a facility that provides for its containment, with the intention of retrieval;
- (liv) "**the Act**" means the National Nuclear Regulator Act, 1999 (Act No. 47 of 1999);
- (lv) "**tissue weighting factor**" means a multiplier of the *equivalent dose* to an organ or tissue used for *radiation protection* purposes to account for the different sensitivities of different organs and tissues to the induction of stochastic effects of *radiation*, the value of which is that specified in the *IAEA International Basic Safety Standards for Protection against Ionizing Radiation and for the Safety of Radiation Sources*;
- (lvi) "**workplace monitoring**" means *monitoring* using measurements made in the working environment.

Section 2

Exclusion, exemption, registration, licensing and clearance

2.1 Exclusion

2.1.1 Exclusion of actions

In terms of the provisions of section 2 (2) (b) of the *Act*, the *Act* does not apply where,

2.1.1.1 the level of *radioactivity* concentration of each *radioactive nuclide* in materials is below –

- (a) 0.2 Bq per gram for artificial *radioactive nuclides*;
- (b) 0.5 Bq per gram for naturally occurring *radioactive nuclides* of uranium and thorium and their progeny except for *radon*;
- (c) 10 Bq per gram for potassium-40 in materials that are used in building construction or *disposed of*;
- (d) 50 Bq per gram for potassium-40 in all other materials; or

2.1.1.2 the level of total *radioactivity* content is below 1000 Bq.

2.1.2 Where the provisions of the *Act* apply to an *action* but the *Regulator* is of the opinion that such *action* is not amenable to regulatory control, the Board must advise the Minister on -

- (a) the publication of a notice determining the *action* as not amenable to regulatory control; and
- (b) appropriate steps which can be taken by the relevant level of Government or any person or body.

2.2 Exemption

2.2.1 Principles

The general principles for the issue of a *certificate of exemption* as contemplated in section 22 (3) (b) (ii) of the *Act* are as follows:

2.2.1.1 the *radiation risk* to individuals caused by the *action* concerned must be sufficiently low not to be of regulatory concern;

2.2.1.2 the collective radiological impact of the *action* concerned must be sufficiently low not to warrant regulatory control in the prevailing circumstances; and

2.2.1.3 the *action* concerned must be inherently safe, with no appreciable likelihood of scenarios that could lead to a failure to meet the criteria in 2.2.1.1 and 2.2.1.2.

2.2.2 *Exemption without further consideration*

Actions involving *radioactive material* will qualify for *exemption* by the *Regulator* without further consideration where the following criteria are fulfilled in all feasible situations:

2.2.2.1 the *effective dose* expected to be incurred by any member of the public due to the *exempted action* is 10 μSv per annum or less; and the *collective effective dose* committed by performing the *action* for one year is no more than 1 person-Sv; or

2.2.2.2 an *assessment* for the optimisation of protection shows that *exemption* is the optimum option; or

2.2.2.3 either the *radioactivity* concentration or the total *radioactivity* content of each *radioactive nuclide* in the *radioactive material* is below the levels specified in Annexure 1 and the quantity possessed or processed in a period of one year is less than one tonne; or

2.2.2.4 the *radioactivity* in the material is associated with naturally occurring *radioactive nuclides* that are not processed for their radioactive, *fissile* or *fertile* properties, and the *effective dose* expected to be incurred by any member of the public due to the *exempted action* is less than 0.25 mSv per annum.

2.2.3 *Exemption with further consideration*

Actions which involve *radioactive material* which do not qualify for *exemption* without further consideration as envisaged in section 2.2.2 can be given further consideration subject to a case-by-case evaluation by the *Regulator* based on the specific *radioactivity*, the total *radioactivity* of discrete *radioactive nuclides* or on *exposure* scenarios.

2.2.4 *Exemption for the transport of radioactive material*

The exemption criteria for the transport of *radioactive material* are those provided for in the *IAEA Regulations for the Safe Transport of Radioactive Material*, applicable in terms of section 4.8.

2.3 *Registration*

Actions other than those that qualify for a *certificate of exemption*, or which require a *nuclear installation licence* or a *nuclear vessel licence*, must be subject to the process of *registration* as contemplated in sections 22 and 23 of the *Act*.

2.4 Licensing

Any *nuclear installation* or nuclear vessel must be subject to the process of licensing as contemplated in sections 21, 23 and 24 of the *Act*.

2.5 Clearance

Radioactive materials which fall within a *Nuclear Installation Licence*, *Nuclear Vessel Licence* or *Certificate of Registration* may be cleared from further compliance with the requirements of the *nuclear authorisation* provided that such materials meet the principles for *exemption* as detailed in 2.2 or that approval has been given by the *Regulator* on a case-by-case consideration.

Section 3

Principal radiation protection and nuclear safety requirements

The following principal *radiation protection* and *nuclear safety* requirements apply to *actions authorised* by, or seeking authorisation in terms of, a *nuclear installation licence*, a *nuclear vessel licence* or a *certificate of registration*.

3.1 Dose and risk limits

3.1.1 The *dose* to an individual arising from normal operating conditions must not exceed the limits specified in Annexure 2.

3.1.2 The *risk* of fatality from any *action* as defined in the *Act* must not exceed the limits specified in Annexure 3.

3.2 Optimisation of radiation protection and nuclear safety

The magnitude of *doses* to individuals, the number of people exposed and the likelihood of incurring exposures must be kept as low as reasonably achievable, economic and social factors being taken into account (ALARA).

3.3 Prior safety assessment

Measures to control the *risk of nuclear damage* to individuals must be determined on the basis of a *prior safety assessment* which is suitable and sufficient to identify all significant *radiation* hazards and to evaluate the nature and expected magnitude of the associated *risks*, with due regard to the *dose* and *risk* limits in Annexures 2 and 3.

Where it can be justified that no credible accident scenarios exist, a *risk assessment* to demonstrate compliance with Annexure 3 is not required to be included as part of the *prior safety assessment*.

3.4 Good engineering practice

Installations, equipment or *plant* requiring a *nuclear installation licence*, a *nuclear vessel licence* or a *certificate of registration* and having an impact on *radiation* or *nuclear safety* must be designed, built and operated in accordance with good engineering practice.

3.5 Safety culture

A *safety culture* must be fostered and maintained to encourage a questioning and learning attitude to *radiation protection* and *nuclear safety* and to discourage complacency.

3.6 Retrospective application of regulations

3.6.1 Subject to 3.6.2, where compliance with the applicable requirements cannot be demonstrated for an *action* which is restricted in terms of section 20 of *the Act* and which existed before the coming into force of these regulations, the person engaged in that *action* must within two months of the coming into force of these regulations or within two months of the issuing of the *nuclear authorisation*, whichever is the later, submit to the *Regulator* an *action plan* to bring the *action* into compliance.

3.6.2 The requirements specified in 4.5.4 do not apply to bulk mineral residue deposits and/or facilities where deposition was discontinued prior to the date of these regulations or prior to the date of such deposits and/or facilities being *authorised* by the *Regulator*, whichever date is the earlier. These facilities must nevertheless still be regulated and the doses must be shown to be optimised.

3.7 Regulatory approval of *radiation protection* and *nuclear safety* measures

3.7.1 The holder of the *nuclear authorisation* is responsible for *radiation protection* and *nuclear safety*, including compliance with applicable requirements such as the preparation of the required *safety assessments*, programmes and procedures relating to the siting, design, construction, operation and *decommissioning* of facilities.

3.7.2 Situations where formal approval of *radiation protection* and *nuclear safety* measures by the *Regulator* is necessary should be limited to those where this is appropriate taking into account the nature and extent of the *risk* and the need for building stakeholder confidence.

3.8 Accident management and *emergency planning, emergency preparedness and emergency response*

Where the *prior safety assessment or operational safety assessment* has identified the reasonable possibility of a *nuclear accident*, accident prevention and mitigation measures based on the principle of *defence in depth* and which address accident management procedures including *emergency planning, emergency preparedness and emergency response* must be established, implemented and maintained. The principle of *defence in depth* must be applied as appropriate.

3.9 *Defence in depth*

A multilayer (*defence in depth*) system of provisions for *radiation protection and nuclear safety* commensurate with the magnitude and likelihood of the potential *exposures* involved shall be applied to *sources* such that a failure at one layer is compensated for or corrected by subsequent layers, for the purposes of –

- (a) preventing *nuclear accidents*;
- (b) mitigating the consequences of any such accidents; and
- (c) restoring *sources* to safe conditions after any such accident.

3.10 Quality management

A quality management programme must be established, implemented and maintained in order to ensure compliance with the conditions of the *nuclear authorisation*.

3.11 Application of *radiation protection and nuclear safety*

The application of the *radiation protection and nuclear safety* requirements contained in these regulations to any *action* should be commensurate with the characteristics of the *Action* and with the magnitude and likelihood of the *exposure*, as determined in the *safety assessments*. Not all the requirements are relevant to every *action*.

Section 4

Requirements applicable to regulated actions

Subject to 4.12, the following requirements apply to *actions authorised by a nuclear installation licence, nuclear vessel licence or certificate of registration*.

4.1 Operational safety assessment

4.1.1 *Operational safety assessments* must be made and submitted to the *Regulator* at intervals specified in the *nuclear authorisation* and which must be commensurate with the nature of the operation and the *radiation risks* involved.

4.1.2 *Operational safety assessments* must be of sufficient scope and must be conducted and maintained in order to demonstrate continuing compliance with the *dose, risk* limits and other relevant conditions of the *nuclear authorisation*.

4.1.3 The *operational safety assessment* must establish the basis for all the *operational safety*-related programmes, limitations and design requirements.

4.2 Controls and limitations on operation

4.2.1 The holder of a *nuclear authorisation* is restricted to the *actions* within the specified *site* and within any limitations imposed in the authorisation.

4.2.2 Technical specifications must be established, implemented and maintained, where applicable, in terms of the *safety assessment*. Such operating technical specifications must provide a link between the *safety assessment* and the operation and must, as a minimum, include the following:

4.2.2.1 operating safety limitations as imposed by the design or by the safety criteria;

4.2.2.2 surveillance requirements to verify that equipment important to safety is operating satisfactorily or that parameters are within the safety limitations; and

4.2.2.3 limitations on the operation, in the event that equipment important to safety becomes inoperable or in the event that safety limitations are exceeded.

4.2.3 *Radioactive waste acceptance criteria* in respect of waste *disposal or storage* facilities must be established.

4.2.4 Operations must be conducted in accordance with formal procedures as required by the conditions of the *nuclear authorisation*.

4.3 Maintenance and inspection programme

4.3.1 An appropriate maintenance and inspection programme must be established.

4.3.2 The maintenance and inspection programme must be implemented to ensure that the reliability and integrity of installations, equipment and *plant* having an impact on *radiation* and *nuclear safety* are commensurate with the *dose limits* and *risk limits* in Annexures 2 and 3.

4.4 Staffing and qualification

4.4.1 An adequate number of competent, qualified and trained staff must be responsible for carrying out the functions associated with *radiation protection* and *nuclear safety* and for maintaining an appropriate *safety culture*.

4.4.2 The appropriate staff must be consulted on all decisions, that may impact on *radiation protection* and *nuclear safety*.

4.5 Radiation protection

4.5.1 Optimisation of protection

Measures commensurate with the magnitude and likelihood of *exposure* must be implemented to ensure that *exposures* associated with the *authorised action* are kept as low as reasonably achievable, economic and social factors being taken into account (ALARA).

4.5.2 Dose constraints

4.5.2.1 Where applicable in terms of the *prior safety assessment*, the optimisation of *radiation protection* must be subject to *dose constraints* specific to the *authorised action*, which must not exceed values that can cause the relevant *dose limits* to be exceeded and which will ensure as far as practicable that *doses* are restricted by application of the ALARA principle on a *source-specific* basis rather than by *dose limits*.

4.5.2.2 For members of the public, the *dose constraint* applicable to the *average member of the critical group* within the exposed population is 0,25 mSv per year specific to the *authorised action* unless otherwise agreed by the *Regulator* on a case-by-case basis, taking into account the *dose limit* specified in Annexure 2 for *exposure* of members of the public from all *sources*.

4.5.3 Annual authorised discharge quantity

The *Regulator* may, for the purposes of controlling radioactive *discharges* from a single *authorised action*, determine a *source-specific annual authorised discharge quantity* in the *nuclear authorisation*, which must take into account the *dose constraint* contemplated in section 4.5.2.2.

4.5.4 Radiation dose limitation

The *normal operational exposure* of individuals must be restricted to ensure that neither the *effective dose* nor the *equivalent dose* to relevant organs or tissues, caused by the possible combination of *authorised actions*, exceeds any relevant *dose limit* specified in Annexure 2. In order to comply with these regulations holders of *nuclear authorisations* must, as a precondition for engagement of occupationally exposed workers who are not their employees, obtain from the employers, including self employed individuals, the previous *occupational exposure* history of such workers.

4.5.5 Medical Surveillance and Health Register

4.5.5.1 A comprehensive medical surveillance programme and health register must be established and maintained for all occupationally exposed workers in a form approved by the NNR. All entries in the health register must be made by an appointed medical practitioner or a person so authorized in writing. The holder must retain the register for a period of 40 years from the date of last entry.

4.5.5.2 An employee must have right of access to his medical records and health register at all times.

4.5.5.3 After consent has been obtained from the employee, the holder must provide the NNR with access to the employee's medical records and health register. The NNR may, with the consent of the employee, appoint an independent medical practitioner to assist in the conduct of a review of said records.

4.5.6 Dose register

A *dose register* of every occupationally exposed worker must be established and maintained.

4.6 *Radioactive waste management*

4.6.1 A *radioactive waste* management programme must be established, implemented and maintained in order to –

4.6.1.1 ensure waste generation control;

4.6.1.2 ensure the identification, quantification, characterisation and classification of any *radioactive waste* generated;

4.6.1.3 provide for the necessary treatment and other waste management steps leading to safe clearance, or *authorised discharge, disposal*, reuse or recycling; and

4.6.1.4 provide for the safe storage of *radioactive waste* between any waste management processes.

4.6.2 The safety of long-term *radioactive waste* storage options must be assured for the envisaged period of storage.

4.6.3 *Radioactive material*, radioactively contaminated material or radioactive waste may be removed from further compliance with the conditions of the *nuclear authorisation* if such material is transported to the *site* of any other *authorised action* or complies with the requirements for –

4.6.3.1 an *authorised discharge*; or

4.6.3.2 *authorised recycling* or *authorised reuse*; or

4.6.3.3 *clearance*; or

4.6.3.4 the material is transported directly to an *authorised waste storage* or *disposal* facility and complies with the applicable waste acceptance criteria.

4.7 *Environmental monitoring and surveillance*

An appropriate *environmental monitoring* and surveillance programme must be established, implemented and maintained to verify that the *storage, disposal* or effluent *discharge* of *radioactive waste* complies with the conditions of the *nuclear authorisation*.

4.8 Transport of *radioactive material*

Transport of *radioactive material* or of any equipment or objects contaminated with *radioactive material* off the *site* or on roads which are accessible to the public must be carried out in terms of the provisions of the *IAEA Regulations for The Safe Transport of Radioactive Material*, in the revision specified in the *nuclear authorisation*.

4.9 Physical security

Physical security arrangements must be established, implemented and maintained in order to demonstrate that all necessary measures are taken to prevent, as far as is reasonable, unauthorised access to *sites* or diversion, theft or removal of *radioactive material* that does not meet the requirements for *clearance* in terms of section 2.5.

4.10 Records and reports

4.10.1 A system of record keeping for all records specified in the *nuclear authorisation* must be established, implemented and maintained.

4.10.2 Operational reports must be submitted to the *Regulator* at predetermined periods as specified in the *nuclear authorisation* and must contain such information as the *Regulator* may require on the basis of the *safety assessments*.

4.10.3 A reporting mechanism must be established, implemented and maintained for *nuclear incidents* and *nuclear accidents* or any other events that the *Regulator* may specify in the authorisation.

4.11 Monitoring of workers

4.11.1 In workplaces where workers are liable to receive *doses* exceeding three tenths of the applicable *dose limits* in section 1.1.1 of Annexure 2, as identified by the *prior safety assessment* and confirmed by subsequent *operational safety assessments*, *individual monitoring* of workers must be undertaken where appropriate, adequate and feasible.

4.11.2 For workers in the workplaces contemplated in section 4.11.1 for whom *individual monitoring* is inappropriate, inadequate or not feasible, the *occupational exposure* of such workers must be assessed from the results of *workplace monitoring* and information on the locations and durations of the workers.

4.11.3 In workplaces where occupationally exposed workers are unlikely to receive *doses* exceeding three tenths of the applicable *dose limits* in section 1.1.1 of Annexure 2, *workplace monitoring* must be implemented to keep under review the workplace *exposure* conditions, in order to maintain an awareness of any

significant changes in conditions and to enable *doses* to be assigned to occupationally exposed workers on the basis of general workplace *exposure* conditions.

4.12 Application to *radon exposure*

For *actions* where the *prior safety assessment* contemplated in section 3.3, or the subsequent *workplace monitoring* contemplated in section 4.11.3, demonstrates that the *occupational exposure* to *radon* does not exceed an action level of 6 mSv/a, the requirements of section 4 applicable to *occupational exposure* to *radon* shall be limited to those of sections 4.4, 4.5.5, 4.5.6, 4.10 and 4.11.3.

Section 5

Decommissioning

The following requirements apply to *actions authorised* by a *nuclear installation licence*, *nuclear vessel licence* or *certificate of registration* which involves the *decommissioning* of any installation, *plant* or equipment having an impact on *radiation protection* and *nuclear safety*, or the release of radioactively contaminated land for other uses.

5.1 *Decommissioning strategy and planning*

5.1.1 A *decommissioning* strategy must be submitted as part of the *prior safety assessment* and must be updated throughout the operation of the *authorised action* as a basis for detailed *decommissioning* planning.

5.1.2 A *decommissioning* plan must be submitted to the *Regulator* as a basis for authorisation of specific *actions* or phases of *decommissioning*.

5.1.3 The *decommissioning* plan must specify any *institutional controls* that are required to maintain *radiation* safety after termination of the *period of responsibility* of the holder of the *nuclear authorisation* and must minimise as far as reasonable the need for such *institutional controls*.

5.2 *Availability of resources*

It must be demonstrated to the *Regulator* that sufficient *resources* will be available from the time of cessation of the operation to the termination of the *period of responsibility*.

5.3 Requirements for *decommissioning* operations

All *decommissioning* operations must be conducted in compliance with the applicable requirements of section 4.

5.4 Release of radioactively contaminated land

5.4.1 A *site* used in the conduct of an *authorized action* may be released for unrestricted use provided that it is demonstrated—

5.4.1.1 that radioactive contamination and *radioactive materials* which can reasonably be attributed to the *authorized action* have been removed from the *site* or, in the case of naturally occurring *radioactive nuclides*, have activity concentrations below the levels for *exclusion* specified in section 2.1.1.1 (b), (c) and (d); or

5.4.1.2 where the provisions in section 5.4.1.1 cannot reasonably be achieved, remedial measures have been implemented in accordance with section 4.5.1 to achieve optimization of protection constrained in accordance with section 4.5.2 such that the annual *effective dose* received by the *average member of the critical group* for all feasible future situations, arising from the residual radioactive contamination and *radioactive materials* which can reasonably be attributed to the regulated *action*, does not exceed the *dose constraint* that was applicable during the operations.

5.4.2 In the event that the release of a *site* in accordance with the conditions in section 5.4.1.2 can only be reasonably achieved by imposing restrictions on the use of the *site*, the *Regulator* may, subject to the conditions in section 5.4.1.2 being met, approve the release of that *site* for restricted use.

5.5 Obligations under other statutes

Where there are obligations on the holder of the *nuclear authorisation* under other statutes with respect to *decommissioning*, the requirements under 5.1 and 5.2 may be integrated into an overall *decommissioning* strategy and funding mechanism which may serve to satisfy all relevant statutes in accordance with the co-operative governance agreements established in terms of section 6 of *the Act*.

Section 6

Accidents, incidents and emergencies

The provisions of this section are applicable to emergency *exposure* situations requiring protective *action* to reduce or avert temporary *exposures*.

6.1 Criteria for the definition of a *nuclear accident*

Any occurrence or succession of occurrences having the same origin and resulting in an unintended/unauthorised exposure to radiation or release of *radioactive material*, which is capable of giving rise to an *effective dose* in excess of 1 mSv to the public off-site in a year, or in excess of 50 mSv to a worker on *site* received essentially at the time of the event, is regarded as a *nuclear accident* as defined in section 1 (xiii) of the Act.

6.2 Criteria for the definition of a *nuclear incident*

Any unintended event which is reasonably capable of giving rise to an *effective dose* equal to or in excess of 0,1 mSv to the public off *site* received essentially at the time of the event, or the unintended spread of radioactive contamination or exposure to radiation, which could reasonably give rise to an *effective dose* in excess of 20 mSv to a worker on *site* received essentially at the time of the event, or significant failure of safety provisions, is regarded as a *nuclear incident* as defined in section 1 (xvii) of the Act.

6.3 Information to be supplied

The holder of a *nuclear authorisation* must immediately inform the *Regulator* when a *nuclear accident* occurs or an incident has arisen or is expected to occur or arise, as the case may be, and shall provide such information as may be required, including –

- 6.3.1 the current situation and its evolution;
- 6.3.2 measures taken to terminate the *nuclear accident* and/or incident to protect workers and members of the public; and
- 6.3.3 the *exposures* that have occurred and those expected to be incurred.

6.4 Emergency or remedial measures

Emergency or remedial measures must be considered in the vicinity of a *nuclear accident* where the potential exists that any member of the public may receive more than an annual *effective dose* of 1 mSv resulting from the accident.

Section 7

General

- 7.1 The safety standards referred to in section 1(xiii)(a) of *the Act* are the criteria for a *nuclear accident* specified in 6.1.
- 7.2 The *exclusion* levels provided for in the safety standards referred to in section 2(2)(b) of *the Act* are the *exclusion* criteria specified in 2.1.
- 7.3 The *exemption* criteria specified in the safety standards referred to in section 22(3)(b)(ii) of *the Act* are the *exemption* criteria specified in 2.2.
- 7.4 The safety standards with respect to the *risk of nuclear damage* referred to in section 1(xxii)(a)(iii) of *the Act* are the *risk* levels corresponding to the release criteria contained in 2.5 and 5.4 for materials and land respectively.
- 7.5 The safety standard with respect to the *risk of nuclear damage* referred to in section 37(2)(b) of *the Act* is the potential *dose* criterion specified in 6.4.
- 7.6 The safety standards with respect to the *risk of nuclear damage* referred to in section 39(2) of *the Act* are the *risk* levels corresponding to the release criteria contained in 2.5 and 5.4 for materials and land respectively.
- 7.7 The safety standards referred to in section 41(4)(e) of *the Act* are the release criteria contained in 2.5 and 5.4 for materials and land respectively.
- 7.8 The *exemption* criteria provided for in the safety standards referred to in section 41(4)(f) of *the Act* are the *exemption* criteria specified in 2.2.
- 7.9 The safety standards referred to in section 41(4)(g) of *the Act* are the release criteria contained in 2.5 and 5.4 for materials and land respectively.

Annexure 1**Exempt radioactivity concentrations and exempt total radioactivity content**

<i>Radioactive nuclide</i>	<i>Radioactivity Concentration (Bq/g)</i>	<i>Total Radioactivity Content (Bq)</i>
H-3	1 E+06	1 E+09
Be-7	1 E+03	1 E+07
C-14	1 E+04	1 E+07
O-15	1 E+02	1 E+09
F-18	1 E+01	1 E+06
Na-22	1 E+01	1 E+06
Na-24	1 E+01	1 E+05
Si-31	1 E+03	1 E+06
P-32	1 E+03	1 E+05
P-33	1 E+05	1 E+08
S-35	1 E+05	1 E+08
Cl-36	1 E+04	1 E+06
Cl-38	1 E+01	1 E+05
Ar-37	1 E+06	1 E+08
Ar-41	1 E+02	1 E+09
K-40	1 E+02	1 E+06
K-42	1 E+02	1 E+06
K-43	1 E+01	1 E+06
Ca-45	1 E+04	1 E+07
Ca-47	1 E+01	1 E+06
Sc-46	1 E+01	1 E+06
Sc-47	1 E+02	1 E+06
Sc-48	1 E+01	1 E+05
V-48	1 E+01	1 E+05
Cr-51	1 E+03	1 E+07
Mn-51	1 E+01	1 E+05
Mn-52	1 E+01	1 E+05
Mn-52m	1 E+01	1 E+05
Mn-53	1 E+04	1 E+09
Mn-54	1 E+01	1 E+06
Mn-56	1 E+01	1 E+05
Fe-52	1 E+01	1 E+06
Fe-55	1 E+04	1 E+06
Fe-59	1 E+01	1 E+06
Co-55	1 E+01	1 E+06
Co-56	1 E+01	1 E+05
Co-57	1 E+02	1 E+06
Co-58	1 E+01	1 E+06
Co-58m	1 E+04	1 E+07
Co-60	1 E+01	1 E+05

<i>Radioactive nuclide</i>	<i>Radioactivity Concentration (Bq/g)</i>	<i>Total Radioactivity Content (Bq)</i>
Co-60m	1 E+03	1 E+06
Co-61	1 E+02	1 E+06
Co-62m	1 E+01	1 E+05
Ni-59	1 E+04	1 E+08
Ni-63	1 E+05	1 E+08
Ni-65	1 E+01	1 E+06
Cu-64	1 E+02	1 E+06
Zn-65	1 E+01	1 E+06
Zn-69	1 E+04	1 E+06
Zn-69m	1 E+02	1 E+06
Ga-72	1 E+01	1 E+05
Ge-71	1 E+04	1 E+08
As-73	1 E+03	1 E+07
As-74	1 E+01	1 E+06
As-76	1 E+02	1 E+05
As-77	1 E+03	1 E+06
Se-75	1 E+02	1 E+06
Br-82	1 E+01	1 E+06
Kr-74	1 E+02	1 E+09
Kr-76	1 E+02	1 E+09
Kr-77	1 E+02	1 E+09
Kr-79	1 E+03	1 E+05
Kr-81	1 E+04	1 E+07
Kr-83m	1 E+05	1 E+12
Kr-85	1 E+05	1 E+04
Kr-85m	1 E+03	1 E+10
Kr-87	1 E+02	1 E+09
Kr-88	1 E+02	1 E+09
Rb-86	1 E+02	1 E+05
Sr-85	1 E+02	1 E+06
Sr-85m	1 E+02	1 E+07
Sr-87m	1 E+02	1 E+06
Sr-89	1 E+03	1 E+06
Sr-90*	1 E+02	1 E+04
Sr-91	1 E+01	1 E+05
Sr-92	1 E+01	1 E+06
Y-90	1 E+03	1 E+05
Y-91	1 E+03	1 E+06
Y-91m	1 E+02	1 E+06
Y-92	1 E+02	1 E+05
Y-93	1 E+02	1 E+05
Zr-93*	1 E+03	1 E+07
Zr-95	1 E+01	1 E+06

<i>Radioactive nuclide</i>	<i>Radioactivity Concentration (Bq/g)</i>	<i>Total Radioactivity Content (Bq)</i>
Zr-97*	1 E+01	1 E+05
Nb-93m	1 E+04	1 E+07
Nb-94	1 E+01	1 E+06
Nb-95	1 E+01	1 E+06
Nb-97	1 E+01	1 E+06
Nb-98	1 E+01	1 E+05
Mo-90	1 E+01	1 E+06
Mo-93	1 E+03	1 E+08
Mo-99	1 E+02	1 E+06
Mo-101	1 E+01	1 E+06
Tc-96	1 E+01	1 E+06
Tc-96m	1 E+03	1 E+07
Tc-97	1 E+03	1 E+08
Tc-97m	1 E+03	1 E+07
Tc-99	1 E+04	1 E+07
Tc-99m	1 E+02	1 E+07
Ru-97	1 E+02	1 E+07
Ru-103	1 E+02	1 E+06
Ru-105	1 E+01	1 E+06
Ru-106*	1 E+02	1 E+05
Rh-103m	1 E+04	1 E+08
Rh-105	1 E+02	1 E+07
Pd-103	1 E+03	1 E+08
Pd-109	1 E+03	1 E+06
Ag-105	1 E+02	1 E+06
Ag-110m	1 E+01	1 E+06
Ag-111	1 E+03	1 E+06
Cd-109	1 E+04	1 E+06
Cd-115	1 E+02	1 E+06
Cd-115m	1 E+03	1 E+06
In-111	1 E+02	1 E+06
In-113m	1 E+02	1 E+06
In-114m	1 E+02	1 E+06
In-115m	1 E+02	1 E+06
Sn-113	1 E+03	1 E+07
Sn-125	1 E+02	1 E+05
Sb-122	1 E+02	1 E+04
Sb-124	1 E+01	1 E+06
Sb-125	1 E+02	1 E+06
Te-123m	1 E+02	1 E+07
Te-125m	1 E+03	1 E+07
Te-127	1 E+03	1 E+06
Te-127m	1 E+03	1 E+07

<i>Radioactive nuclide</i>	<i>Radioactivity Concentration (Bq/g)</i>	<i>Total Radioactivity Content (Bq)</i>
Te-129	1 E+02	1 E+06
Te-129m	1 E+03	1 E+06
Te-131	1 E+02	1 E+05
Te-131m	1 E+01	1 E+06
Te-132	1 E+02	1 E+07
Te-133	1 E+01	1 E+05
Te-133m	1 E+01	1 E+05
Te-134	1 E+01	1 E+06
I-123	1 E+02	1 E+07
I-125	1 E+03	1 E+06
I-126	1 E+02	1 E+06
I-129	1 E+02	1 E+05
I-130	1 E+01	1 E+06
I-131	1 E+02	1 E+06
I-132	1 E+01	1 E+05
I-133	1 E+01	1 E+06
I-134	1 E+01	1 E+05
I-135	1 E+01	1 E+06
Xe131m	1 E+04	1 E+04
Xe-133	1 E+03	1 E+04
Xe-135	1 E+03	1 E+10
Cs-129	1 E+02	1 E+05
Cs-131	1 E+03	1 E+06
Cs-132	1 E+01	1 E+05
Cs-134m	1 E+03	1 E+05
Cs-134	1 E+01	1 E+04
Cs-135	1 E+04	1 E+07
Cs-136	1 E+01	1 E+05
Cs-137*	1 E+01	1 E+04
Cs-138	1 E+01	1 E+04
Ba-131	1 E+02	1 E+06
Ba-140*	1 E+01	1 E+05
La-140	1 E+01	1 E+05
Ce-139	1 E+02	1 E+06
Ce-141	1 E+02	1 E+07
Ce-143	1 E+02	1 E+06
Ce-144*	1 E+02	1 E+05
Pr-142	1 E+02	1 E+05
Pr-143	1 E+04	1 E+06
Nd-147	1 E+02	1 E+06
Nd-149	1 E+02	1 E+06
Pm-147	1 E+04	1 E+07
Pm-149	1 E+03	1 E+06

<i>Radioactive nuclide</i>	<i>Radioactivity Concentration (Bq/g)</i>	<i>Total Radioactivity Content (Bq)</i>
Sm-151	1 E+04	1 E+08
Sm-153	1 E+02	1 E+06
Eu-152	1 E+01	1 E+06
Eu-152m	1 E+02	1 E+06
Eu-154	1 E+01	1 E+06
Eu-155	1 E+02	1 E+07
Gd-153	1 E+02	1 E+07
Gd-159	1 E+03	1 E+06
Tb-160	1 E+01	1 E+06
Dy-165	1 E+03	1 E+06
Dy-166	1 E+03	1 E+06
Ho-166	1 E+03	1 E+05
Er-169	1 E+04	1 E+07
Er-171	1 E+02	1 E+06
Tm-170	1 E+03	1 E+06
Tm-171	1 E+04	1 E+08
Yb-175	1 E+03	1 E+07
Lu-177	1 E+03	1 E+07
Hf-181	1 E+01	1 E+06
Ta-182	1 E+01	1 E+04
W-181	1 E+03	1 E+07
W-185	1 E+04	1 E+07
W-187	1 E+02	1 E+06
Re-186	1 E+03	1 E+06
Re-188	1 E+02	1 E+05
Os-185	1 E+01	1 E+06
Os-191	1 E+02	1 E+07
Os-191m	1 E+03	1 E+07
Os-193	1 E+02	1 E+06
Ir-190	1 E+01	1 E+06
Ir-192	1 E+01	1 E+04
Ir-194	1 E+02	1 E+05
Pt-191	1 E+02	1 E+06
Pt-193m	1 E+03	1 E+07
Pt-197	1 E+03	1 E+06
Pt-197m	1 E+02	1 E+06
Au-198	1 E+02	1 E+06
Au-199	1 E+02	1 E+06
Hg-197	1 E+02	1 E+07
Hg197m	1 E+02	1 E+06
Hg-203	1 E+02	1 E+05
Tl-200	1 E+01	1 E+06
Tl-201	1 E+02	1 E+06

<i>Radioactive nuclide</i>	<i>Radioactivity Concentration (Bq/g)</i>	<i>Total Radioactivity Content (Bq)</i>
Tl-202	1 E+02	1 E+06
Tl-204	1 E+04	1 E+04
Pb-203	1 E+02	1 E+06
Pb-210*	1 E+01	1 E+04
Pb-212*	1 E+01	1 E+05
Bi-206	1 E+01	1 E+05
Bi-207	1 E+01	1 E+06
Bi-210	1 E+03	1 E+06
Bi-212*	1 E+01	1 E+05
Po-203	1 E+01	1 E+06
Po-205	1 E+01	1 E+06
Po-207	1 E+01	1 E+06
Po-210	1 E+01	1 E+04
At-211	1 E+03	1 E+07
Rn-220*	1 E+04	1 E+07
Rn-222*	1 E+01	1 E+08
Ra-223*	1 E+02	1 E+05
Ra-224*	1 E+01	1 E+05
Ra-225	1 E+02	1 E+05
Ra-226*	1 E+01	1 E+04
Ra-227	1 E+02	1 E+06
Ra-228*	1 E+01	1 E+05
Ac-228	1 E+01	1 E+06
Th-226*	1 E+03	1 E+07
Th-227	1 E+01	1 E+04
Th-228*	1 E+00	1 E+04
Th-229*	1 E+00	1 E+03
Th-230	1 E+00	1 E+04
Th-231	1 E+03	1 E+07
Th-nat* (incl.Th-232)	1 E+00	1 E+03
Th-234*	1 E+03	1 E+05
Pa-230	1 E+01	1 E+06
Pa-231	1 E+00	1 E+03
Pa-233	1 E+02	1 E+07
U-230*	1 E+01	1 E+05
U-231	1 E+02	1 E+07
U-232*	1 E+00	1 E+03
U-233	1 E+01	1 E+04
U-234	1 E+01	1 E+04
U-235*	1 E+01	1 E+04
U-236	1 E+01	1 E+04
U-237	1 E+02	1 E+06
U-238*	1 E+01	1 E+04

<i>Radioactive nuclide</i>	<i>Radioactivity Concentration (Bq/g)</i>	<i>Total Radioactivity Content (Bq)</i>
U-nat*	1 E+00	1 E+03
U-239	1 E+02	1 E+06
U-240	1 E+03	1 E+07
U-240*	1 E+01	1 E+06
Np-237*	1 E+00	1 E+03
Np-239	1 E+02	1 E+07
Np-240	1 E+01	1 E+06
Pu-234	1 E+02	1 E+07
Pu-235	1 E+02	1 E+07
Pu-236	1 E+01	1 E+04
Pu-237	1 E+03	1 E+07
Pu-238	1 E+00	1 E+04
Pu-239	1 E+00	1 E+04
Pu-240	1 E+00	1 E+03
Pu-241	1 E+02	1 E+05
Pu-242	1 E+00	1 E+04
Pu-243	1 E+03	1 E+07
Pu-244	1 E+00	1 E+04
Am-241	1 E+00	1 E+04
Am-242	1 E+03	1 E+06
Am-242m*	1 E+00	1 E+04
Am-243*	1 E+00	1 E+03
Cm-242	1 E+02	1 E+05
Cm-243	1 E+00	1 E+04
Cm-244	1 E+01	1 E+04
Cm-245	1 E+00	1 E+03
Cm-246	1 E+00	1 E+03
Cm-247	1 E+00	1 E+04
Cm-248	1 E+00	1 E+03
Bk-249	1 E+03	1 E+06
Cf-246	1 E+03	1 E+06
Cf-248	1 E+01	1 E+04
Cf-249	1 E+00	1 E+03
Cf-250	1 E+01	1 E+04
Cf-251	1 E+00	1 E+03
Cf-252	1 E+01	1 E+04
Cf-253	1 E+02	1 E+05
Cf-254	1 E+00	1 E+03
Es-253	1 E+02	1 E+05
Es-254	1 E+01	1 E+04
Es-254m	1 E+02	1 E+06
Fm-254	1 E+04	1 E+07
Fm-255	1 E+03	1 E+06

*Parent nuclides and their progeny included in secular equilibrium are listed in the following:

Sr-90	Y-90
Zr-93	Nb-93m
Zr-97	Nb-97
Ru-106	Rh-106
Cs-137	Ba-137m
Ba-140	La-140
Ce-144	Pr-144
Pb-210	Bi-210, Po-210
Pb-212	Bi-212, Tl-208 (0.36), Po-212 (0.64)
Bi-212	Tl-208 (0.36), Po-212 (0.64)
Rn-220	Po-216
Rn-222	Po-218, Pb-214, Bi-214, Po-214
Ra-223	Rn-219, Po-215, Pb-211, Bi-211, Tl-207
Ra-224	Rn-220, Po-216, Pb-212, Bi-212, Tl-208(0.36), Po-212(0.64)
Ra-226	Rn-222, Po-218, Pb-214, Bi-214, Po-214, Pb-210, Bi-210, Po-210
Ra-228	Ac-228
Th-226	Ra-222, Rn-218, Po-214
Th-228	Ra-224, Rn-220, Po-216, Pb-212, Bi-212, Tl-208 (0.36), Po-212 (0.64)
Th-229	Ra-225, Ac-225, Fr-221, At-217, Bi-213, Po-213, Pb-209
Th-nat	Ra-228, Ac-228, Th-228, Ra-224, Rn-220, Po-216, Pb-212, Bi-212, Tl-208 (0.36), Po-212 (0.64)
Th-234	Pa-234m
U-230	Th-226, Ra-222, Rn-218, Po-214
U-232	Th-228, Ra-224, Rn-220, Po-216, Pb-212, Bi-212, Tl-208 (0.36), Po-212 (0.64)
U-235	Th-231
U-238	Th-234, Pa-234m
U-nat	Th-234, Pa-234m, U-234, Th-230, Ra-226, Rn-222, Po-218, Pb-214, Bi-214, Po-214, Pb-210, Bi-210, Po-210
U-240	Np-240m
Np-237	Pa-233
Am-242m	Am-242
Am-243	Np-239

Annexure 2

Dose limits

1. Occupational exposure

1.1 General dose limits

The *occupational exposure* of any worker shall be so controlled that the following limits are not exceeded:

1.1.1 an (average) *effective dose* of 20 mSv per year averaged over five consecutive years;¹

1.1.2 a (maximum) *effective dose* of 50 mSv in any single year;

1.1.3 an *equivalent dose* to the lens of the eye of 150 mSv in a year; and

1.1.4 an *equivalent dose* to the extremities (hands and feet) or the skin of 500 mSv in a year.

1.1.5 in special circumstances, provided that *radiation protection* in the *action* has been optimised as required by 4.5.1 of the regulations but *occupational exposures* still remain above the *dose limit* in 1.1.2 above, the *Regulator* may approve a temporary change in the *dose limit* subject to the agreement of the affected employees, through their representatives where appropriate, and provided that all reasonable efforts are being made to improve the working conditions to the point where compliance with the *dose limits* can be achieved. This temporary change shall not exceed 5 years and shall not be renewed.

1.2 Apprentices and students

For apprentices of 16 to 18 years of age who are training for employment involving *exposure to radiation* and for students of age 16 to 18 who are required to use *sources* in the course of their studies, the *occupational exposure* shall be so controlled that the following limits are not exceeded:

1.2.1 an *effective dose* of 6 mSv in a year;

1.2.2 an *equivalent dose* to the lens of the eye of 50 mSv in a year; and

1.2.3 an *equivalent dose* to the extremities or the skin of 150 mSv in a year.

¹The start of the averaging period shall be coincident with the first day of the relevant annual period starting from the date of entry into force of the Regulations, with no retroactive averaging.

1.3 Women

The annual *effective dose limit* for women of reproductive capacity is the same as that which is generally specified for *occupational exposure* under 1.1 above. Following declaration of pregnancy, a limit on the *equivalent dose* to the abdomen of 2 mSv for the remainder of the pregnancy applies.

1.4 Emergencies

In the event of an emergency or when responding to an accident, a worker who undertakes emergency measures may be exposed to a *dose* in excess of the annual *dose limit* for persons occupationally exposed as specified in 1.1 –

- 1.4.1 for the purpose of saving life or preventing serious injury;
- 1.4.2 if undertaking *actions* intended to avert a large *collective dose*; or
- 1.4.3 if undertaking *actions* to prevent the development of catastrophic conditions.

Under any of the circumstances referred to in 1.4.2 or 1.4.3 above, all reasonable efforts must be made to keep *doses* to the worker below twice the maximum annual *dose limit*. In respect of life-saving interventions as contemplated in 1.4.1 above, every effort shall be made to keep *doses* below ten times the maximum annual *dose limit*. In addition, workers undertaking interventions which may result in their *doses* approaching or exceeding ten times the annual *dose limit* may only do so when the benefits to others clearly outweigh their own *risk*.

2. *Exposure* of visitors and non-occupationally exposed workers at *sites*

The annual *effective dose limit* for visitors to the *sites* and those not deemed to be occupationally exposed is 1 mSv. The annual *dose* equivalent limit for individual organs and tissues of such persons is 10 mSv.

3. *Public exposure*

3.1 The annual *effective dose limit* for members of the public from all *authorised actions* is 1 mSv.

3.2 No *action* may be *authorised* which would give rise to any member of the public receiving a *radiation dose* from all *authorised actions* exceeding 1 mSv in a year.

Annexure 3**Probabilistic *risk* limits**

PUBLIC	
Average annual population <i>risk</i>	10 ⁻⁸ fatalities per year per <i>site</i> (one fatality per one hundred million per year per <i>site</i>)
Maximum annual individual <i>risk</i>	5 x 10 ⁻⁶ fatalities per year (one fatality per two hundred thousand per year)
WORKERS	
Average annual <i>risk</i> to workers	10 ⁻⁵ fatalities per year per <i>site</i> (one fatality per one hundred thousand per year per <i>site</i>)
Maximum annual individual <i>risk</i> to workers	5 x 10 ⁻⁵ fatalities per year per <i>site</i> (one fatality per twenty thousand per year per <i>site</i>)



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ARE YOU READY FOR THE 13-DIGIT ISBN?

PRETORIA, Gauteng. – April 06, 2006. Currently aligned with the worldwide trend, the South African book publishing and related industries are preparing themselves to be ready to increase the International Standard Book Number (ISBN), with effect from 1 January 2007.

The well-known 10-digit ISBN is changing to a 13-digit number. The nature of this change involves the EAN (European Article Number) bar code. In compliance with retail systems a 13-digit bar code is normally assigned to each product in the supply chain. To ensure compatibility between the 10-digit ISBN and the 13-digit international product coding system for bar codes, this unique identifier for a book is prefixed by the "978" EAN product code for books, followed by the 10-digit ISBN, of which the last character - i.e. the check digit, is recalculated according to the preceding 12 characters, eg.:

ISBN 0-624-02126-2 (10 digits)	becomes	ISBN 978-0-624-02126-1 (13 digits)
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The result is a 13-digit number which is the "Bookland EAN" bar code, and in which the 10-digit ISBN is embedded. The 13-digit ISBN is identical to the Bookland EAN bar code.

As a consequence of electronic publishing and other changes in the publishing industry, the system is running out of numbers. To resolve this problem, the length of the ISBN is augmented from 10- to 13-digits. This solution is beneficial in two ways: books already carry both the 10-digit ISBN and its 13-digit equivalent in the bar code. Secondly, it also enables the ISBN system to make use of a new "979" EAN prefix (once the "978" is exhausted) which was reserved years ago for the future use of the book trade within the EAN system.

During the transition period between 2005 and 2007, while the 10-digit system is still officially in place, South Africa will follow the International ISBN Agency's *Guidelines for the implementation of 13-digit ISBNs*. The ISN Agency (National Library of South Africa,

Pretoria) still assigns 10-digit ISBNs, but provides a means to convert these to 13-digit ISBNs.

However, from 1 January 2007 when the 13-digit ISBN as specified by the revised *ISO 2108:2005 – Information and Documentation: International Standard Book Number* is the official ISBN, the publishing scene changes accordingly:

- the ISN Agency assigns only 13-digit ISBNs;
- publishers print only 13-digit ISBNs on their publications, and the 13-digit ISBN replaces the 10-digit ISBN above the bar code, while the 13-digit EAN is displayed below the bar code;
- within systems – now adjusted to accommodate ISBN-13, the 13-digit ISBN is input in the valid ISBN field, while the 10-digit ISBN is displayed in the field for an invalid ISBN – the ISBN standard now specifies a 13-character ISBN, and no longer a 10-digit ISBN;
- publications are ordered by quoting the 13-digit ISBN;
- publishers continuously consult with GS1 South Africa regarding bar codes for their publications.

It is important that between 2005 and 2007 everyone in the supply chain who currently uses an ISBN should review all their existing systems – manual and electronic, develop an action plan and allocate resources to ensure that those systems can accommodate the new 13-digit ISBN well in advance of the January 2007 implementation date. In South Africa the target date is October 2006, with some time left until December 2006 to finalise any outstanding issues.

This change affects all types of publishers (trade, educational, etc.) as well as distributors, retailers, libraries and any other organizations that record and exchange ISBN in automated areas. Although the conversion to the 13-digit primarily affects systems, it also has an impact on editorial processes, sales and marketing, design and production departments, royalty and accounting functions, as well as catalogue systems.

For further information on the 13-digit ISBN, please consult the following persons, and/or URLs:

- Magret Kibido, tel.: 012-401-9718, e-mail: magret.kibido@nlsa.ac.za
- Tienie de Klerk, tel.: 012-401-9731, e-mail: tienie.deklerk@nlsa.ac.za
- Karen Feiling, tel: (011) 789 5777, e-mail: karen.feiling@qs1za.org
- ISBN-13 in SA: <http://www.nlsa.ac.za/isbn13.html>
- Frequently asked questions about changes on the ISBN:
<http://www.lac-bac.gc.ca/iso/tc46sc9/isbn.htm>
- Guidelines for the implementation of 13-digit ISBNs:
<http://www.isbn-international.org/en/download/implementation-guidelines-04.pdf>



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 en inligtingsentrums, verspreiders, boekuitgewersbedryf-kleinhandelketting

IS JY GEREED VIR DIE 13-SYFER-ISBN?

PRETORIA, Gauteng. – 05 April 2006. In ooreenstemming met die neiging/tendens wêreldwyd, berei die Suid-Afrikaanse boekuitgewersbedryf en verwante Industrieë hulle tans voor om die Internasionale Standaardboeknommer (ISBN) met ingang van 1 Januarie 2007 te vergroot.

Die welbekende 10-syfer-ISBN verander na 'n 13-syfer-nommer. Die aard van hierdie verandering betrek die EAN (Europese Artikelnummer) -staafkode. Ter voldoening aan kleinhandelstelsels, word 'n 13-syfer-staafkode normaalweg aan elke produk in die voorsieningsketting toegeken. Om versoenbaarheid te verseker tussen die 10-syfer-ISBN en die internasionale 13-syfer-produkkoderingstelsel vir staafkodes, word die "978"-EAN-produkkode vir boeke vooraan hierdie unieke identifiseerder vir 'n boek geplaas, gevolg deur die 10-syfer-ISBN waarvan die laaste karakter (d.i. die kontrolesyfer) ooreenkomstig die voorafgaande 12 karakters herbereken word, bv.:

ISBN 0-624-02126-2 (10 syfers)	word	ISBN 978-0-624-02126-1 (13 syfers)
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Die gevolg is 'n 13-syfernommer, wat die Bookland EAN-staafkode is, en wat die 10-syfer-ISBN vervat. Die 13-syfer-ISBN is identies aan die Bookland EAN-staafkode.

As gevolg van elektroniese publiserings en ander veranderinge binne die uitgewersbedryf is die nommers in die stelsel besig om op te raak. Om hierdie probleem op te los, word die grootte van die ISBN uitgebrei van 10 tot 13 syfers. Hierdie oplossing is in twee opsigte voordelig: altwee die 10-syfer-ISBN en sy 13-syfer-ekwivalent word reeds in die staafkode van boeke vervat. Tweedens stel dit ook die ISBN-stelsel in staat om (sodra die "978" uitgeput is) gebruik te maak van 'n nuwe "979"-EAN-voorvoegsel, wat jare gelede vir toekomstige gebruik vir die boekhandel binne die EAN-stelsel gereserveer is.

In die oorgangstydperk tussen 2005 en 2007, terwyl die 10-syferstelsel steeds amptelik in gebruik is, sal Suid-Afrika die Internasionale ISBN-agentskap se *Guidelines for the implementation of 13-digit ISBNs* volg. Die Internasionale Standaardnommer (ISN)-agentskap (Nasionale Biblioteek van Suid-Afrika, Pretoria) ken steeds 10-syfer-ISBN's toe, maar voorsien 'n geleentheid om hierdie nommers na 13-syfer-ISBN's om te skakel.

Vanaf 1 Januarie 2007, wanneer die 13-syfer-ISBN, soos deur die hersiene

ISO 2108:2005 – Information and Documentation: International Standard Book Number gespesifiseer, die amptelike ISBN is, verander die uitgewerstoneel egter dienoreenkomstig:

- die ISN-agentskap ken slegs 13-syfer-ISBN's toe;
- uitgewers druk slegs 13-syfer-ISBN's op hulle publikasies, en die 13-syfer-ISBN vervang die 10-syfer-ISBN bokant die staafkode, terwyl die 13-syfer-EAN onder die staafkode verskyn;
- binne stelsels, nou aangepas om die ISBN-13 te akkommodeer, word die 13-syfer-ISBN in die geldige ISBN-veld ingevoer, terwyl die 10-syfer-ISBN in die veld vir 'n ongeldige ISBN verskyn- die ISBN-standaard spesifieer nou 'n 13-karakter-ISBN, en nie meer 'n 10-syfer-ISBN nie;
- publikasies word bestel deur die 13-syfer-ISBN te verstrek;
- uitgewers raadpleeg deurlopend GS1 South Africa (Global Standard 1 South Africa, voorheen bekend as EAN South Africa) betreffende staafcodes vir hulle publikasies.

Dit is belangrik dat elkeen in die voorsieningsketting, wat tans 'n ISBN gebruik, tussen 2005 en 2007 al hulle bestaande stelsels (hand- en elektroniese stelsels) hersien, 'n aksieplan ontwikkel en bronne toewys om te verseker dat daardie stelsels die nuwe 13-syfer-ISBN lank voor die Januarie 2007-implementeringsdatum kan akkommodeer. In Suid-Afrika is die teikendatum Oktober 2006, met nog tyd oor tot Desember 2006 om enige uitstaande sake af te handel.

Hierdie verandering raak alle soorte uitgewers (handel, opvoedkundig, ens.) asook verspreiders, kleinhandelaars, biblioteke en enige ander organisasies wat ISBN's in geoutomatiseerde velde dokumenteer en uitruil. Hoewel die omskakeling na die 13-syfer-ISBN hoofsaaklik stelsels raak, het dit ook 'n uitwerking op redaksionele prosesse, verkope en bemaking, ontwerp- en produksie-afdelings, tantième- en rekenkundige funksies, asook katalogiseerstelsels.

Vir meer inligting oor die 13-syfer-ISBN's, raadpleeg die volgende persone en/of URL's:

- Magret Kibido, tel.: 012-401-9718, e-pos: magret.kibido@nlsa.ac.za
- Tienie de Klerk, tel.: 012-401-9731, e-pos: tienie.deklerk@nlsa.ac.za
- ISBN-13 in SA: <http://www.nlsa.ac.za/isbn13.html>
- Karen Feiling, tel: (011) 789 5777, e-pos: karen.feiling@gs1za.org
- Algemene vrae oor veranderinge aan die ISBN:
<http://www.lac-bac.ac.za/iso/tc46sc9/isbn.htm>
- Riglyne vir die implementering van die 13-syfer-ISBN's:
<http://www.isbn-international.org/en/download/implementation-guidelines-04.pdf>

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